## Appendix 2

## **ACRONYMS**

ACSNI Advisory Committee on the Safety of Nuclear Installations

AECB Atomic Energy Control Board

AIM Abnormal Incidents Manual

ALARA As Low as Reasonably Achievable

ANSI American National Standards Institute

ASDV Atmospheric Steam Discharge Valve

ASME American Society of Mechanical Engineers

BFP Boiler Feed Pump

BPC Boiler Pressure Control

C-6 Consultative Document 6

CANDU CANada Deuterium Uranium

CCW Condenser Cooling Water

CEO Chief Executive Officer

CNSC Canadian Nuclear Safety Commission

COG CANDU Owners' Group

CRO Control Room Operator

CSA Canadian Standards Association

CSDV Condenser Steam Discharge Valve

CSP Critical Safety Parameter

DBE Design Basis Earthquake

DEL Derived Emission Limit

ECCS Emergency Core Cooling System

ECI Emergency Coolant Injection

ECIS Emergency Coolant Injection System

EPS Emergency Power Supply

EQ Environmental Qualification

EWS Emergency Water Supply

FP Full Power

GSS Guaranteed Shutdown State

HPECI High Pressure Emergency Coolant Injection

HPES Human Performance Enhancement System

HT Heat Transport

HTS Heat Transport System

1AEA International Atomic Energy Agency

1CRP International Commission for Radiological Protection

1FB Irradiated Fuel Bay

1M Impairments Manual

INPO Institute of Nuclear Power Operations

INSAG International Nuclear Safety Advisory Group

IUFWT Inter-unit Feedwater Tie

KEA Key Effectiveness Area

KI Potassium Iodide

LOCA Loss of Coolant Accident

LOR Loss of Regulation

LORA Loss of Regulation Accident

LZCS Liquid Zone Control System

MCCR Ministry of Consumer and Commercial Relations

MISA Municipal and Industrial Strategy for Abatement

MOEE Ministry of Environment and Energy

MOSG Ministry of the Solicitor General

NPP Nuclear Power Plant

NRC Nuclear Regulatory Commission (USA)

OE Operating Experience

O&M Operating and Maintenance

OGSS Overpoisoned Guaranteed Shutdown State

OP&P Operating Policies and Principles

PHT Primary Heat Transport

PHTS Primary Heat Transport System

PRA Probabilistic Risk Assessment

PROL Power Reactor Operating License

PT Pressure Tube

QA Quality Assurance

## 22107.A2 - Acronyms

| <b>Q</b> C | Quality Control                            |
|------------|--|
| RRS        | Reactor Regulating System                  |
| SDM        | Safety Design Matrix                       |
| SDS        | Shutdown System                            |
| SDS1       | Shutdown System Number One                 |
| SDS2       | Shutdown System Number Two                 |
| SDSE       | Shutdown System Enhancement (Pickering-A)  |
| SER        | Significant Event Report                   |
| SG         | Steam Generator (Boiler)/Standby Generator |
| SOE        | Safe Operating Envelope                    |
| SRV        | Steam Release Valve (at Pickering)         |
| SS         | Shift Supervisor / Shift Superintendent    |
| SSS        | Special Safety System                      |
| TMI        | Three Mile Island                          |
| TSP        | Trip Set Point                             |
| WANO       | World Association of Nuclear Operators     |

Prepared by:

L. Haacke

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